Chemical Engineering 612

Reactor Design and Analysis

Lecture 6 Criticality Theory



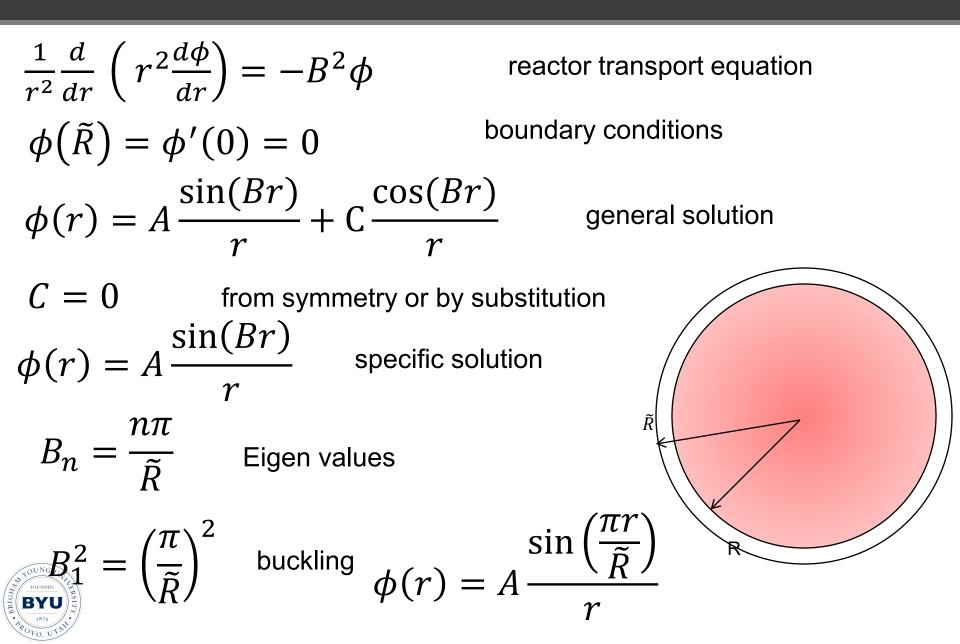
Spiritual Thought

D&C 101: 16

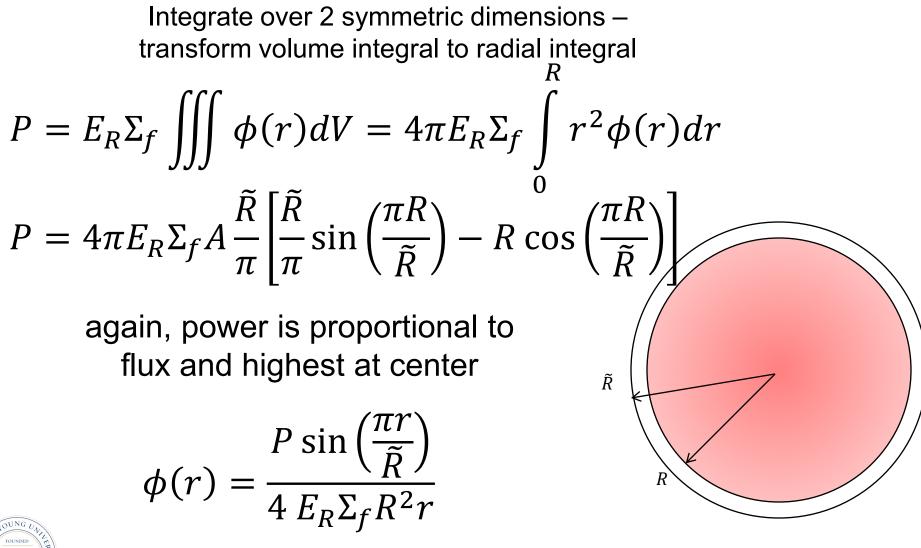
Therefore, let your hearts be comforted concerning Zion; for all flesh is in mine hands; be still and know that I am God.



Spherical Reactor



Spherical Reactor Power





Infinite Cylindrical Reactor

$$\frac{1}{r}\frac{d}{dr}\left(r\frac{d\phi}{dr}\right) = -B^{2}\phi = \frac{d^{2}\phi}{dr^{2}} + \frac{1}{r}\frac{d\phi}{dr}$$

$$\phi(\tilde{R}) = \phi'(0) = 0; |\phi(r)| < \infty$$

$$\frac{d^{2}\phi}{dr^{2}} + \frac{1}{r}\frac{d\phi}{dr} + \left(B^{2} - \frac{m^{2}}{r^{2}}\right)\phi = 0$$

$$\phi(r) = AJ_{0}(Br) + CY_{0}(Br)$$

$$B_{n} = \frac{x_{n}}{\tilde{R}}$$

$$For the second seco$$

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reactor transport equation

boundary conditions

zero-order (m=0) Bessel equation

general solution involves Bessel functions of first and second kind

flux is finite

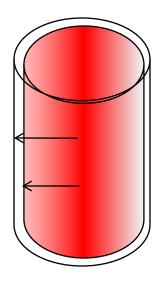
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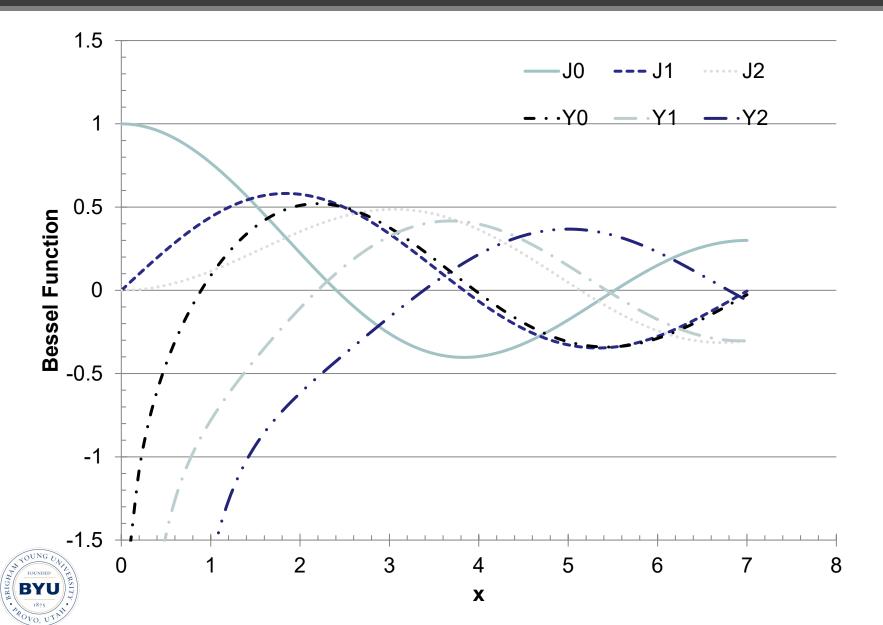
roots of Bessel functions - ϕ is zero at boundary $ilde{R}$

first root

solution (power roduction determines A)



Bessel Functions



Infinite Cylindrical Reactor Power

$$P = E_R \Sigma_f \iiint \phi(r) dV = 2\pi E_R \Sigma_f \int_0^R r \phi(r) dr \quad \begin{array}{l} \text{transform volume} \\ \text{integral to radial} \\ \text{integral -} \\ \text{becomes power} \\ \text{per unit length} \end{array}$$

$$P = \frac{2\pi E_R \Sigma_f R^2 A J_1(2.405)}{2.405} \qquad R \quad P = \frac{2\pi E_R \Sigma_f R^2 A J_1(2.405)}{2.405} \qquad R \quad P = \frac{0.738P}{E_R \Sigma_f R^2} J_0 \left(\frac{2.405r}{R}\right)$$



again, power is proportional to power and highest at center

Finite Cylindrical Reactor

$$\frac{1}{r}\frac{\partial}{\partial r}\left(\frac{\partial \phi}{\partial r}\right) + \frac{\partial^2 \phi}{\partial z^2} = -B^2 \phi \qquad \text{reactor transport equation}$$

$$\phi\left(\tilde{R}, z\right) = \phi'(0, z) = \phi\left(r, \frac{\tilde{H}}{2}\right) = \phi\left(r, -\frac{\tilde{H}}{2}\right) = 0 \qquad \text{boundary conditions}$$

$$\phi\left(r, z\right) = R(r)Z(z) \qquad \text{separation of variables}$$

$$\frac{1}{r}\frac{\partial}{\partial r}\left(\frac{\partial \phi}{\partial r}\right) + \frac{\partial^2 \phi}{\partial z^2} = \frac{Z}{r}\frac{\partial}{\partial r}\left(\frac{\partial R}{\partial r}\right) + R\frac{\partial^2 Z}{\partial z^2}R = -B^2R(r)Z(z)$$

$$\frac{1}{R}\frac{\partial}{\partial r}\left(\frac{\partial R}{\partial r}\right) + \frac{1}{Z}\frac{\partial^2 Z}{\partial z^2}R = -B^2$$

$$\text{since } R \text{ and } Z \text{ vary independently, both portions of the equation must equal (generally different) constants, designated as B_R and B_Z , respectively$$

Η

 \mathbf{F}

Finite Cylinder Solution

solution is the product of the infinite cylinder and infinite slab solutions Buckling is higher than for either the infinite plane or the infinite cylinder. Buckling generally increases with increasing leakage, and there are more surfaces to leak here than either of the infinite cases.

Neutron Flux Contours

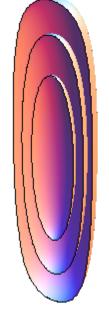
Neutron flux in finite cylindrical reactor



3D contours of neutron flux at high power



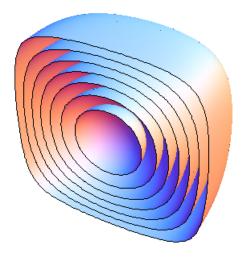


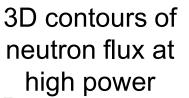


3D contours of neutron flux at low power

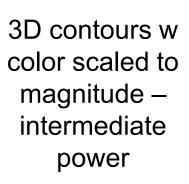
Neutron Flux Contours

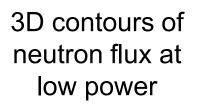
Neutron flux in finite parallelepiped (cubical) reactor





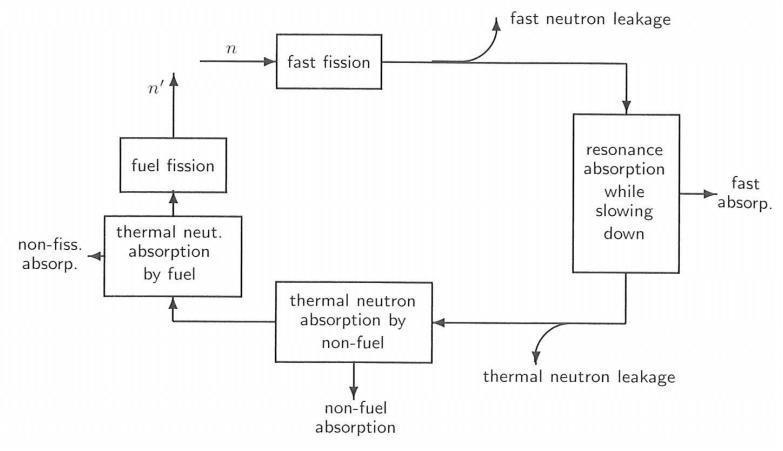






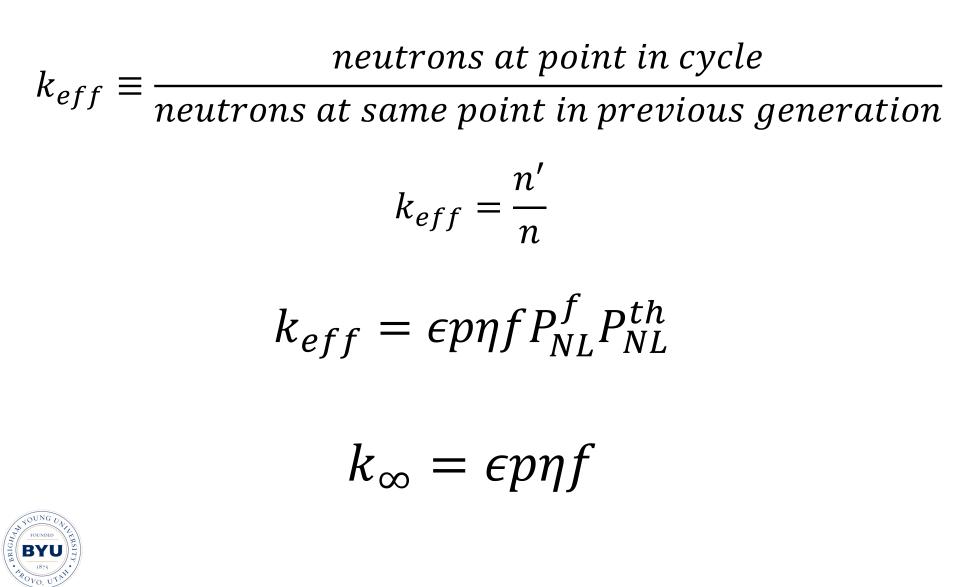
Fast Neutron Life Cycle

• What happens to fast neutrons?





Multiplication Factor



Reactor Considerations

• Increase Power?

$$k_{eff} > 1$$

• Decrease Power?

$$k_{eff} < 1$$

- Most Reactors have K_{eff}>1, but cancel excess out with absorptive "poisons", which are removed with time.
- Reactors designed to not reach prompt supercriticality
- If k_{eff} increases, "feedback" effects resist increase
- What if we want to change amount of fuel or moderator?
 - Impacts various "six factor" parameters
 - Changes k_{eff}



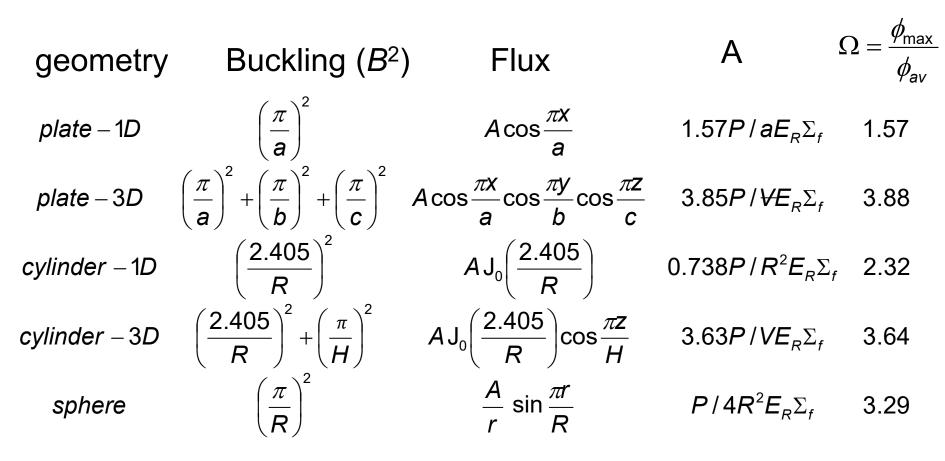
One-Group Six Factor Formula

$$n' \equiv n\epsilon p\eta f P_{NL}^f P_{NL}^{th}$$

- n' next generation neutrons
- *n* neutrons produced per thermal fission
- ϵ ratio of total neutrons to thermal neutrons (1.0-1.08)
- *p* resonance escape probability (0.8-0.9)
- $f \qquad \text{thermal utilization} = \frac{\Sigma_a^F \phi^F V^F}{\Sigma_a^F \phi^F V^F + \Sigma_a^{NF} \phi^{NF} V^{NF}} (0-1)$ $\eta \qquad \text{fission factor} = v \frac{\Sigma_f^F}{\Sigma_a^F} (2.0-2.2)$
- P_{NL}^{f} non-leakage of fast neutrons = $\exp(-B_{c}^{2}\tau)$ (near 1)

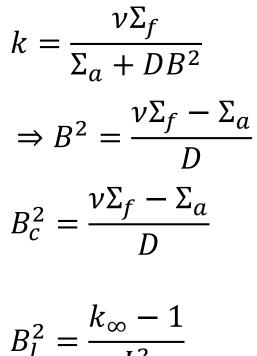


Bare Reactor Summary





Critical Buckling



value of k for critical reactor

value of B when k = 1

critical material buckling

$$B_l^2 = \frac{k_\infty - 1}{L^2}$$

 $\frac{\nu\Sigma_f - \Sigma_a}{D} = \frac{k_\infty - 1}{I^2}$

geometric buckling

geometric and material buckling must be equal for a critical reactor



Resonance Escape Probability

 Probability of neutron slowing down without being absorbed

_ thermalized neutrons

neutrons that slow

- *p* < 1
- Highly dependent on resonance region

•
$$p = \prod_{i} p_{i} =$$

 $e^{\left(-\frac{N_{F}V_{F}}{\xi \sum_{sM}V_{M}} \sum_{i} \int \frac{\sigma_{a}(E) \ dE}{1 + \sigma_{a}(E) / \sigma_{o}^{E}}\right)}$
 $\approx e^{\left(-\frac{N_{F}V_{F}}{\xi \sum_{sM}V_{M}} I_{eff}\right)}$

